



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 12, 1993

Docket No. 50-320

Dr. Robert L. Long
Vice President and Director
Corporate Services and TMI-2 Divisions
GPU Nuclear Corporation
Post Office Box 480
Middletown, Pennsylvania 17057-0191

Dear Dr. Long:

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2 - ISSUANCE OF
AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE NO. DPR-73
(IAC NO. M81878)

The Commission has issued the enclosed Amendment No. 44 to Facility Operating License No. DPR-73 for the Three Mile Island Nuclear Station, Unit 2, in response to your letter dated August 1, 1991 (Technical Specification Change Request No. 68). Your letter dated August 1, 1991, also requested a change in Table 4.3-3 of the Recovery Operations Plan. The staff has determined that this change is no longer applicable since Table 4.3-3 was transferred to the offsite Dose Calculation Manual (ODCM) by TMI-2 Recovery Operations Plan Change approval 44. We have verified that the surveillance requirement in your August 1, 1991 amendment request has been incorporated in your ODCM, Part II, Table 2.3-1.

Amendment No. 44 modifies the Appendix A Technical Specifications by substituting numerical criteria to limit Processed Water Disposal System (PWDS) operations and effluents in lieu of the current technical specification which requires NRC review and approval of operating procedures to limit PWDS operations and effluents.

A copy of the related Safety Evaluation supporting Amendment No. 44 is enclosed. Notice of Issuance will be included in the Commission biweekly Federal Register notice.

Sincerely,

Lee H. Thonus, Project Manager
Non-Power Reactors and Decommissioning
Project Directorate
Division of Operating Reactor Support
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 44 to DPR-73
2. Safety Evaluation

cc w/enclosures:
See next page

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GPU Nuclear Corporation Unit No. 2

cc:

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Three Mile Island Nuclear Station
Docket No. 50-320

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555-0001

GPU NUCLEAR CORPORATION

DOCKET NO. 50-320

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 44
License No. DPR-73

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by GPU Nuclear Corporation (the licensee), dated August 1, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-73 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 44, are hereby incorporated in the license. GPU Nuclear Corporation shall operate the facility in accordance with the Technical Specifications and all Commission Orders issued subsequent to March 28, 1979.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactors and Decommissioning
Project Directorate
Division of Operating Reactors Support
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications Appendix A

Date of Issuance: July 12, 1993

ATTACHMENT TO LICENSE AMENDMENT NO. 44

FACILITY OPERATING LICENSE NO. DPR-73

DOCKET NO. 50-320

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

ii
1-6
3.9-3

B 3/4 9-1
B 3/4 9-2
6-11

Insert

ii
1-6
3.9-3
3.9-4
B 3/4 9-1
B 3/4 9-2
6-11

DEFINITIONS

SECTION	PAGE
1.0 DEFINITIONS - (Continued)	
1.21 CONTAINMENT ISOLATION	1-6
1.22 OFFSITE DOSE CALCULATION MANUAL (ODCM)	1-6
1.23 MEMBER(S) OF THE PUBLIC	1-6
1.24 UNRESTRICTED AREA	1-6
1.25 SITE BOUNDARY	1-6
1.26 BASE CASE WATER	1-6
FACILITY MODES (TABLE 1.1)	1-7
FREQUENCY NOTATION (TABLE 1.2)	1-8
2.0 SAFETY LIMITS	2-1

1.21 CONTAINMENT ISOLATION shall exist when:

- a. Each penetration is:
 1. Closed by an accessible manual valve, a welded or bolted blind flange, or a deactivated automatic valve secured in the closed position to provide isolation of each penetration, or;
 2. Open per an approved procedure but can be closed pursuant to Specification 1.21.a.1. Controls shall be implemented to minimize the time the penetration is allowed open and to specify the conditions for which the penetration is open. Penetrations shall be expeditiously closed upon completion of the conditions specified in the approved procedures.
- b. The Equipment Hatch is closed and sealed.
- c. Each Containment Airlock is OPERABLE pursuant to Specification 3.6.1.6.

1.22 The OFFSITE DOSE CALCULATION MANUAL (ODCM) shall contain the methodology and parameters used in the calculation of off-site doses resulting from radioactive gases and liquid effluents, in the calculation of gaseous and liquid effluent monitoring Alarm/Trip Setpoints, and in the conduct of the Environmental Radiological Monitoring Program. The ODCM shall also contain (1) the Radioactive Effluent Controls and Radiological Environmental Monitoring Programs required by Section 6.8.4 and (2) descriptions of the information that should be included in the Annual Radiological Environmental Operating Report and the Annual Radioactive Effluent Release Report required by Specifications 6.9.1.1 and 6.9.1.2, respectively.

1.23 MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the GPU System, GPU contractors or vendors. Also excluded from this category are persons who enter the site to service equipment or to make deliveries.

1.24 UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation and radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional, and/or recreational purposes.

1.25 SITE BOUNDARY shall be that line beyond which the land is neither owned, nor leased, nor otherwise controlled by GPU Nuclear.

1.26 BASE CASE WATER is ACCIDENT GENERATED WATER that contains the average activity levels provided in Table 2.2 of the Supplement 2 to the PEIS (NUREG-0683). These activity levels form the basis for the NRC staff's analysis of the environmental impacts of evaporator discharge.

AUXILIARY BUILDING AIR CLEANUP EXHAUST SYSTEM

3.9.12.2 The Auxiliary Building Air Cleanup Exhaust System shall be OPERABLE with one of the four system air cleanup exhaust fans OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTION:

With the Auxiliary Building Air Cleanup Exhaust System inoperable, restore the system to operable status within 4 hours or suspend all operations involving movement of liquid and solid radioactive wastes in the Auxiliary Building (other than sampling evolutions required by the Technical Specifications or RECOVERY OPERATIONS PLAN), the release of which could exceed 50% of the Appendix B Technical Specification instantaneous release rate for gaseous effluent, until the system is restored to OPERABLE status.

ACCIDENT GENERATED WATER

3.9.13 When processing ACCIDENT GENERATED WATER through the Processed Water Disposal System (PWDS) Vaporizer, concentrations of radioactive contaminants (except tritium) in the vapor vented to the atmosphere shall, on a quarterly average, be less than or equal to the activity levels in Table 3.9-1. These activity levels equate to 0.1% of the concentrations for BASE CASE WATER.

APPLICABILITY: For the duration of ACCIDENT GENERATED WATER processing

ACTION:

With the release rate(s) exceeding the above limit(s), or if any operating condition exists which would prevent the quarterly average activity levels to be maintained below those listed in Table 3.9-1, the environmental release will be terminated and corrective action taken.

TABLE 3.9-1

PROCESSED WATER DISPOSAL SYSTEM ENVIRONMENTAL
RELEASE RATES ($\mu\text{Ci}/\text{ml}$)

Constituent	Environmental Release Rate Limit
Cesium-137	3.7E-8
Cesium-134	8.8E-10
Strontium-90	1.1E-7
Antimony-125/Tellurium-125m	2.3E-9
Carbon-14	1.0E-7
Technetium-99	1.0E-9
Iron-55	4.8E-10
Cobalt-60	4.8E-10
Iodine-129	<6.0E-10
Cerium-144	<1.8E-9
Manganese-54	<4.0E-11
Cobalt-58	<4.0E-11
Nickel-63	<6.0E-10
Zinc-65	<9.8E-11
Ruthenium-106/Rhodium-106	<3.3E-10
Silver-110m	<5.6E-11
Promethium-147	<4.8E-9
Europium-152	<3.8E-13
Europium-154	<4.4E-11
Europium-155	<1.1E-10
Uranium-234	<1.0E-11
Uranium-235	<1.2E-11
Uranium-238	<1.2E-11
Plutonium-238	<1.2E-11
Plutonium-239	<1.4E-11
Plutonium-240	<1.4E-11
Plutonium-241	<6.5E-10
Americium-241	<1.2E-11
Curium-242	<1.0E-10

BASES

3.4.9.1 SPENT FUEL STORAGE POOL "A" WATER LEVEL MONITORING

Spent Fuel Storage Pool "A" Water Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the Spent Fuel Storage Pool "A."

3.4.9.2 SPENT FUEL STORAGE POOL "A" WATER LEVEL

The water level in the Spent Fuel Storage Pool "A" has been established to limit the dose rate, due to the storage of Canisters, to acceptable levels.

3.4.9.3 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL MONITORING

Fuel Transfer Canal Water Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the deep end of the Fuel Transfer Canal.

3.4.9.4 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL

The water level in the Fuel Transfer Canal (deep end) has been established to limit the dose rate, due to the storage of the plenum assembly and Canisters, to acceptable levels.

3.4.9.12 FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEM

The requirement for the Fuel Handling Building/Auxiliary Building Air Cleanup System to be operating or OPERABLE ensures that radioactive material released to these buildings will be filtered through the HEPA filters prior to release to the atmosphere. In the event the systems are not restored to OPERABLE status within 4 hours, the Technical Specifications require the suspension of any liquid or solid radioactive waste handling, the release of which could exceed 50% of the instantaneous release rate limits for gaseous effluents specified in Section 2.1.2 of the Appendix B Technical Specifications. These restrictions correspond to solid radioactive waste with a total activity of greater than 1 curie of particulates with half-lives greater than eight (8) days and liquid radioactive waste with an activity greater than 0.5 curies of particulates with half-lives greater than eight (8) days.

3.4.9.13 ACCIDENT GENERATED WATER

The analyses in Supplement 2 of the PEIS (NUREG-0683) assumed processing BASE CASE WATER with a vaporizer discharge to the atmosphere containing $\leq 0.1\%$ of the radioactive particulates contained in the PWDS influent. Thus, the PEIS release values for effluent discharge have been established as the Limiting Condition for Operation.

BASES

The Processed Water Disposal System (PWDS) is designed to operate, in the coupled mode, with an overall system Decontamination Factor (DF) of at least 1000 for particulates. That is, the ratio of the concentration of a constituent in the influent stream to the concentration of the same constituent in the effluent vapor stream is equal to or greater than 1000. To comply with PEIS (NUREG-0683) limits, periodic sampling is conducted to verify that system DF is at least 1000. This sampling ensures that the effluents are within 0.1% of BASE CASE WATER concentrations, which are listed in Table 3.9-1.

When operating the vaporizer in the decoupled mode, the vaporizer feed shall be limited to concentrations of contaminants such that the resulting effluent shall be less than 1/1000 of BASE CASE WATER concentrations for all radionuclides except tritium.

Tritium is not included in Table 2.2 of the PEIS (NUREG-0683), Supplement 2, since the PWDS does not remove tritium. Tritium releases have been evaluated by the licensee in Section 5.1 of the PWDS Technical Evaluation Report (TER), which has been reviewed and approved by the NRC staff.

The activity releases resulting from PWDS processing and discharge of BASE CASE WATER are a small fraction of the releases permissible in accordance with existing regulatory requirements. Even though higher levels of contaminant releases are permissible in accordance with 10 CFR 20 and the LMI 2 Appendix B Technical Specifications, and would be of very minor environmental consequence, the PWDS shall be operated in a manner such that the PEIS (NUREG-0683) projections of environmental impact are not exceeded.

6.8.2.1 Each procedure and any change to any procedure prepared pursuant to 6.8.1, shall be prepared, reviewed and approved in accordance with 6.5 and will be reviewed periodically as required by ANSI 18.7 - 1976.

6.8.2.2 Deleted.

6.8.3.1 Temporary changes to procedures of 6.8.1 may be made provided that:

- a. The intent of the original procedure control is not altered, and
- b. (1) For those procedures which affect the operational status of unit systems or equipment, the change is approved by two members of the unit management staff, at least one of whom holds a Senior Reactor Operator's License. (Note: The requirement for a Senior Reactor Operator's License applies during Mode 1 only.) If one of the two above signatures is not by a supervisory person within the Department having cognizance of the procedure being changed, the signature of that supervisory person within the department will also be required, or

(2) For those procedures which do not affect the operational status of unit systems or equipment, the change is approved by two members of the responsible organization. If one of the two above signatures is not by a section manager/director within the Department having cognizance of the procedure being changed, the signature of that section manager/director within the department will also be required, and
- c. The change is documented, Independent Safety Review completed, and the required reviews and approvals are obtained within 14 days.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as is reasonably achievable. The program (1) shall be contained in the ODCM, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- (1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 44 TO FACILITY OPERATING LICENSE

NO. DPR-73

GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2

DOCKET NO. 50-320

1.0 INTRODUCTION

By letter dated August 1, 1991, (reference 1) GPU Nuclear Corporation (GPUN or the licensee) requested the approval of a license amendment to change the Three Mile Island Nuclear Station (TMI-2) Appendix A Technical Specifications (TS). The purpose of the amendment request is to replace the current technical specification which requires NRC staff review of operating procedures to limit Processed Water Disposal System (PWDS) operations and effluents with a new technical specification that specifies numerical limits on PWDS effluents.

2.0 BACKGROUND

GPUN submitted a proposal to dispose of contaminated water resulting from the March 1979 accident in July of 1986 (revised October 1986). This water is defined in the TMI-2 TS as accident generated water (AGW). The NRC staff completed an environmental impact review of this proposal and published Supplement 2 to the Programmatic Environmental Impact Statement (PEIS) in June of 1987 (reference 2). On February 25, 1987, the licensee requested a TS change which would allow implementation of the proposal and operation of the PWDS. The NRC staff issued the license amendment and approved the PWDS Technical Evaluation Report (TER) in a letter dated September 11, 1989 (reference 3).

As discussed in the above NRC staff letter (reference 3) the principal technical issues concerning the PWDS evaporator system were:

1. The ability to preprocess water to achieve the base case radionuclide concentrations as described in the PEIS Supplement 2 (reference 2).
2. The ability of the evaporator system to achieve a decontamination factor of 1000 while processing base case water.

3. The ability of the licensee to monitor effluents from the process stack and the building ventilation during routine and off normal conditions.
4. Potential accidents associated with the use of the evaporator.
5. Potential for any safety problems in the transporting of evaporator bottoms to the low level waste disposal site.

The staff found that the PWDS system design and operational parameter limits would satisfactorily address each of the issues. NRC staff approval was required on the initial PWDS operating procedures and all procedure changes which affected disposal of AGW.

In reviewing operating procedures and procedure changes, the NRC staff verified that the procedures and process controls would achieve effluents less than 0.1 percent of base case water and that they would assure adequate monitoring of effluents. (The last two items related to accidents and transportation are addressed in the PWDS TER, annual updates to the PWDS TER, and NRC approval of these documents.) The NRC staff also performed a special inspection of the licensee program for sampling and analysis of process and effluent streams from the PWDS, including independent laboratory measurements taken by the NRC staff (reference 4). The staff concluded that procedures and analytical capabilities for sampling and measuring process and effluent streams were acceptable.

3.0 EVALUATION

The change to the IS is principally administrative in nature. Operating limits on the PWDS would be placed directly in the IS rather than using NRC staff review and approval to verify that they were incorporated into operating procedures. Limits on effluent releases to the environment would not change.

The staff evaluation of each of the licensee proposed changes to the IMI-2 technical specifications are as follows:

- (1) Change IS "INDEX" by revising page ii of the table of contents to reflect the addition of the definition of "BASE CASE WATER."

Evaluation: The licensee has updated page ii to reflect the addition of a definition of "BASE CASE WATER" as discussed previously. The staff finds this change administrative in nature and therefore finds it acceptable.

- (2) Add to IS section 1.0 "DEFINITIONS" the definition for "BASE CASE WATER."

Evaluation: The definition of BASE CASE WATER is identical to that used by the NRC staff in their environmental impact analysis in the PEIS Supplement 2 (reference 2). The staff finds this change acceptable.

- (3) Change TS section 3.9.13 "ACCIDENT GENERATED WATER" to delete the requirement for using NRC-approved procedures for disposal of ACCIDENT GENERATED WATER and substitute an effluent limit of less than 0.1 percent of BASE CASE WATER on a quarterly average basis.

Evaluation: This change removes the requirement for NRC staff approval of procedures, but places the limits the staff would impose directly in the TS. The staff finds this change acceptable.

- (4) Change TS section 3.9 to add table 3.9-1 "PROCESSED WATER DISPOSAL SYSTEM ENVIRONMENTAL RELEASE RATES ($\mu\text{Ci}/\text{ml}$)"

Evaluation: This table is added to implement the revised wording of TS section 3.9.13. The tabular values are 0.1 percent of the values in table 2.2 of PEIS supplement 2 (reference 2). The NRC staff environmental evaluation in PEIS supplement 2 and the staff approval of the PWDS (reference 3) were based on environmental effluent releases not exceeding 0.1 percent of table 2.2 on a quarterly average basis. The staff finds this change acceptable.

- (5) Change the basis for TS section 3/4.9.13 to remove the outdated bases which refer to the need for the then impending environmental impact evaluation and Commission action and add a new basis describing the results of the staff environmental evaluation and the Commission action.

Evaluation: The revised basis reflects the completion of the NRC staff PEIS, Supplement 2 and approval of GPUN PWDS TER. It also reflects the Commission action in April of 1989 to approve processing and disposal of AGW. The staff finds these changes acceptable. Note: The bases are not part of the TS as defined in 10CFR50.36.

- (6) Delete TS section 6.8.3.1(d). This section described the licensee methodology for making temporary changes to procedures requiring NRC approval.

Evaluation: Since procedures related to the disposal of AGW will no longer be required to have NRC approval, this TS section is moot and no longer needed. The staff finds this change acceptable.

Based on the above evaluations, we find that the licensee proposed changes to the TS are consistent with the NRC staff PEIS, Supplement 2, prior staff approval of the PWDS TER, and Commission approval of disposal of AGW. The staff therefore finds the proposed changes acceptable.

4.0 STATE CONSULTATION

In accordance with Commission regulations, the cognizant individual of the State of Pennsylvania was notified of the proposed issuance of this amendment. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment involves changes in administrative procedures and changes in reporting requirements. The NRC staff has determined that the amendment involves no increase in the amounts, and no change in the types, of any effluents that may be released offsite, and that there is no change in individual cumulative occupational exposure or exposure to the public. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 16225 dated March 25, 1993). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

6.0 CONCLUSIONS

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and (3) the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

7.0 REFERENCES

1. GPUN letter C312-91-2046 dated August 1, 1991, from R. L. Long to NRC with attached Technical Specification Change Request No. 68 and Recovery Operations Plan Change Request No. 47
2. Final Programmatic Environmental Impact Statement related to decontamination and disposal of radioactive wastes resulting from March 28, 1979 accident at Three Mile Island Nuclear Station, Unit 2, Supplement 2 (NUREG 0683, Supplement 2), June 1987
3. NRC letter dated September 11, 1989, from M. T. Masnik to M. B. Roche, GPUN re: Issuance of Amendment (TAC NO. 62068) and Approval of the TER on Processed Water Disposal System
4. NRC letter with attached Inspection Report No. 50-320/91-02 dated March 8, 1991, from J. J. Joyner, III to R. L. Long, GPUN

Principal Contributor: Lee H. Thonus

Date: July 12, 1993